

COLLEGE OF ENGINEERING

School of Nuclear Science and Engineering

Engineering Expo Presentation

Multiphysics Modeling of a Pebble Bed Reactor

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Presentation Outline

1: Background (Slides 3-7)

- Advanced Reactors
- TRISO Fuel
- Pebble Bed Reactors

- 2: Significance (Slides 8-19)
 - Homogenized Diffusion
 - Pebble Tracking Transport (PTT)
 - Packing Fraction

- 3: Design Challenges (Slides 20-23)
 - Processing DEM Data
 - Making Assumptions
 - Running the Simulation





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Background Reactor Generations

- Reactor designs are split into generations
- The majority of reactors in america are generation II and III
- The industry is currently working towards Generation IV
- Gen. IV has many advantages:
 - Cost Effective
 - Inherent and Enhanced safety
 - Proliferation Resistant
- Pebble-Bed Modular Reactor (PBMR)

Generation IV: Nuclear Energy Systems Deployable no later than 2030 and offering significant advances in sustainability, safety and reliability, and economics



Background TRISO Fuel

- TRISO (TRI-structural ISOtropic) fuel starts with enriched uranium (UO₂) reactor fuel kernel
 - Fuel kernels are covered in multiple layers of graphite and ceramics
- The final product is a fuel sphere, these spheres have many advantages over rod forms of reactor fuel:
 - High temperature resistance, Contains the fission products, Structurally sound, and Nearly impossible to extract the uranium





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[Cogliati, 2006]

Background Pebble Bed

- Pebble bed reactors use TRISO fuel spheres within the reactor core
- The uranium inside these fuel spheres causes a reaction which produces heat
- Helium gas is pushed through the gaps in between the pebbles to remove heat

Background Pebble Bed Reactor

- The reactor for this project is a pebble-bed reactor
- Fuel pebbles are contained within the annular region of the reactor core
- Annular means ring shaped cylinder
- We have restricted our model to just the area outlined in blue







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Significance Comparison Between Two Methods

- This project compares two reactor core modeling methods
- The assumptions made in each model affect the neutron physics (neutronics)
- The first model is of a homogeneous core
 - Physically unrealistic but proven and provides acceptable results
- The second is a core using a Pebble-Tracking Transport (PTT) algorithm
 - Physically realistic but unproven and computationally expensive



Significance Pebble Packing Factor

- Packing fraction is the amount of space occupied by the pebbles within a certain volume.
- Important to understand fluctuations throughout the core so we can determine its impact on the reactor performance.



Packing Factor

Volume of pebbles Volume containing pebbles



E.g.

Significance Homogeneous Model

 Assumes a single homogenized material for the core; a mixture of all the pebble materials into one "blob"





[Cogliati, 2006]



Significance Pebble Tracking Transport Model

- Tracks each pebble individually, this is much more realistic
- More precision in neutronics modeling for the core over the homogenized method



Individual pebbles within the core region modeled as nodes



Significance Accident Scenarios to Compare Methods

- Two accident scenarios will be used in the comparison between modeling methods
- Accident scenarios for this project:
 - Before and after a seismic event (earthquake)
 - Depressurized Loss of Forced Coolant (DLOFC)
- These scenarios represent design-basis accidents individually and beyond design-basis accidents when coupled
 - Important for risk assessment and safety analysis for regulators like the Nuclear Regulatory Commision (NRC)



Significance Accident Scenario: Seismic Event

- This accident scenario will simulate
 an earthquake
- The event will shake the core and force the pebbles to become more densely packed and therefore a higher packing fraction
- As the fuel is more densely packed, the reactivity of the core will increase which leads to increased power and temperatures



[Chen et al., 2020]



Significance Accident Scenario: Depressurized Loss of Forced Coolant

- Depressurized Loss of Forced Coolant (DLOFC)
- In this scenario we simulate the aftermath of an accident which the reactor loses its coolant, i.e. coolant is no longer flowing through the core
- Core materials including the fuel will begin to heat up significantly
- Optimal upper limit for fuel is 1600 C



[Strydom et al., 2010]





- For regulators it is important to have a comprehensive code that can represent both normal operation and an accident scenario, and MAMMOTH is promising.
- MOOSE is the INL Multi-Physics Framework that allows for the construction and solving of Partial Differential Equations (PDEs) that represent physical system behavior.
- MAMMOTH is a MOOSE Module specifically designed for simulating Reactors.
- Capable of simulating both a reactor in steady state and with transients (constant conditions vs. changing).
- A goal of this project is to evaluate the performance of MAMMOTH for PBRs.
- MAMMOTH takes an input file that includes the mesh of our core and a set of properties which we define to the core regions.



Significance CUBIT/TETGEN

- Meshing softwares used for homogenized and the PTT base model respectively.
- CUBIT utilizes an Add-On developed by INL specifically for this style of core that optimizes the mesh for neutronics of reactors.
- TetGen is used to handle the large amount of elements for the PTT meshes, can even model individual elements.
- Both are methods that are commonly used by the industry and research professionals.



Individual pebbles within the core region



Significance PEBBLES

- Developed by Dr. Cogliati at INL, DEM code specifically for modeling pebble distribution.
- All fuel spheres are treated as individual elements, with their own center location and a set physical properties.
- At a starting point the real life forces acting on a pebble are converted to equations the code can interpret.
- At subsequent time steps, resultant equations from the system and collisions of elements.



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Significance What Will These Models Show?

- Each accident scenario model will show neutronics or heat conduction results.
- On the right is a MAMMOTH output for a pre-seismic Homogenized Diffusion.
- Left part of the image represents thermal flux, right side power density.



Design Challenges

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Design Challenges Preparing Input Files

- List of 450,000 pebble center coordinates
- Need to visualize packing fraction distribution throughout the core



Design Challenges Making Simplifications

- We are not going to be able to model the core perfectly.
- The key is knowing what assumptions you can reasonably make.





Design Challenges Running the Simulation

- Important to know what your ٠ code (MAMMOTH) is actually doing
- Know what to modify between ٠ our three scenarios
- How do we implement packing ٠ factor?

Pebble-bed effective thermal conductivity:

$$(1 - \epsilon) p_s C_{p,s} \frac{dT_s}{dt} - \nabla \cdot \kappa_s T_s + \alpha (T_s - T_f) + \dot{q}_s = 0$$

Porosity Temperature of Solid

(Related to packing factor)

(What we are solving for)

Neutron Diffusion Equation:

$$\frac{1}{v}\frac{\partial\varphi}{\partial t} = s - \Sigma_a\varphi + D\nabla^2\varphi$$





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References 1/2



- 1. Chen, X., Dai, Y., Yan, R., Mei, M., Zhang, J., Zou, Y., & Cai, X. (2020). Experimental study on the vibration behavior of the pebble bed in PB-FHR. *Annals of Nuclear Energy*, *139*, 107193. <u>https://doi.org/10.1016/j.anucene.2019.107193</u>
- 2. Cogliati, J., Ougouag, Abderrafi M. (2006). PEBBLES: A computer code for modeling Packing, Flow, and recirculation of pebbles in a pebble bed reactor. U.S. DOE Idaho National Laboratory.
- 3. Cisneros, Tomas A., (2013). Pebble bed reactors design optimization methods and their application to the pebble bed fluoride Salt cooled reactor. University of California, Berkeley.
- 4. Cundall, P. A., & Strack, O. D. L. (1979). A discrete numerical model for granular assemblies. *Geotechnique*, 29(1), 47–65. https://doi.org/10.1680/geot.1979.29.1.47
- 5. Durst, P. C., Beddingfield, D., Boyer, B., Bean, R., Collins, M., Ehinger, M., Hanks, D., Moses, D. L., & Refalo, L. (2009). Nuclear Safeguards Considerations for the Pebble Bed Modular Reactor (PBMR). U.S. DOE Idaho National Laboratory, Report # INL/EXT-09-16782. https://inldigitallibrary.inl.gov/sites/sti/sti/4374060.pdf
- 6. International Atomic Energy Agency (IAEA). (2011). Status report 70-Pebble Bed Modular Reactor (PBMR). *International Atomic Energy Agency*. https://aris.iaea.org/PDF/PBMR.pdf
- 7. Kaszynski, A. (2020). tetgen 0.5.1. PyPi. https://pypi.org/project/tetgen/#description
- 8. Kruggel-Emden, H., Rickelt, S., Wirtz, S., & Scherer, V. (2008). A study on the validity of the multi-sphere Discrete Element Method. *Powder Technology*, *188*(2), 153–165. https://doi.org/10.1016/j.powtec.2008.04.037
- 9. Mulder, E. J., & Boyes, W. A. (2020). Neutronics characteristics of a 165 MWth Xe-100 reactor. *Nuclear Engineering and Design*, *357*, 110415. https://doi.org/10.1016/j.nucengdes.2019.110415
- 10. Nuclear Energy Agency (NEA). (2013). PBMR Coupled Neutronics/Thermal-Hydraulics Transient Benchmark: The PBMR-400 Core Design. *Nuclear Energy Agency NEA/NSC/DOC(2013)10.*

https://www.oecd-nea.org/jcms/pl_19318/pbmr-coupled-neutronics/thermal-hydraulics-transient-benchmark-the-pb mr-400-core-design-volume-1-the-benchmark-definition

References 2/2



11. Nuclear Energy Research Advisory Committee (NERAC), Generation IV International Forum (GIF). (2002). A Technology Roadmap for Generation IV Nuclear Energy Systems. U.S. DOE Nuclear Energy Research Advisory Committee and the Generation IV International Forum.

https://www.gen-4.org/gif/upload/docs/application/pdf/2013-09/genivroadmap2002.pdf

- 12. Ougouag, A. M., Ortensi, J., & Hiruta, H. (2009). Analysis of an Earthquake-Initiated-Transient in a PBR 2009 International Conference on Advances in Mathematics, Computational Methods, and Reactor Physics ANALYSIS OF AN EARTHQUAKE-INITIATED-TRANSIENT IN A PBR. American Nuclear Society.
- 14. Strydom, G., Reitsma, F., Ivanov, K., Ngeleka, P. T., & Ivanov, K. N. (2010). THE OECD/NEA/NSC PBMR 400 MW COUPLED NEUTRONICS THERMAL HYDRAULICS TRANSIENT BENCHMARK: TRANSIENT RESULTS. *PHYSOR 2010 – Advances in Reactor Physics to Power the Nuclear Renaissance.* https://doi.org/10.13140/2.1.4626.8164
- 14. Strydom, G. (2008). TINTE transient results for the OECD 400 MW PBMR benchmark. *ICAPP 2008 Paper 8180.* https://www.researchgate.net/publication/266672982
- 15. Strydom, G. (2004). TINTE Uncertainty Analysis of the Maximum Fuel Temperature During a DLOFC Event for the 400 MW Pebble Bed Modular Reactor. *ICAPP 2004 Paper 4165.* https://www.researchgate.net/publication/236370888
- 16. Suikkanen, H., Ritvanen, J., Jalali, P., & Kyrki-Rajamäki, R. (2014). Discrete element modelling of pebble packing in pebble bed reactors. *Nuclear Engineering and Design, 273, 24–32.* https://doi.org/10.1016/j.nucengdes.2014.02.022
- 17. Venter, P. J., Mitchell, M. N., & Fortier, F. (2005). PBMR REACTOR DESIGN AND DEVELOPMENT. 18th International Conference on Structural Mechanics in Reactor Technology.

https://repository.lib.ncsu.edu/bitstream/handle/1840.20/31858/S02_2.pdf?sequence=1

- 18. Wang, Y., Ortensi, J., Schunert, S., & Laboure, V. (2018). A Pebble Tracking Transport Algorithm for Pebble Bed Reactor Analysis. *PHYSOR 2018: Reactor Physics Paving The Way Towards More Efficient Systems*, *April*, 1110–1124.
- 19. Yang, X., Gui, N., Tu, J., & Jiang, S. (2014). 3D DEM simulation and analysis of void fraction distribution in a pebble bed high temperature reactor. *Nuclear Engineering and Design*, *270*, 404–411. https://doi.org/10.1016/j.nucengdes.2014.02.010